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FRANCIS, M.	L	١.,
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SHULER, K.

WARD, D.A. WIEMELT, K.

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SECRET

UNCLASSIFIED

CONFIDENTIAL

CORRES.CONTROL

ADMIN RECRD/T130G

CLASSIFICATION

AUTHORIZED CLASSIFIER

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SIGNATURE: The

SILLS, S. SNYDER, D.P. SWARTZ, J.M. KAISER-HILL COMPANY

JUN - 7 2005

05-RF-00555

Gary Morgan, Functional Lead Cadre Project Management Division DOE, RFPO

RSOP FOR COMPONENT REMOVAL, SIZE REDUCTION, AND DECONTAMINATION ACTIVITIES NOTIFICATION LETTER FOR BUILDING 891 COMPONENT REMOVAL, SIZE REDUCTION, AND DECONTAMINATION – DWF-045-05

Attached is a draft transmittal letter to the Colorado Department of Public Health and Environment for the RSOP notification for Building 891 component removal, size reduction and decontamination. The draft transmittal letter has been prepared from DOE RFCA coordinator to CDPHE RFCA coordinator.

Please contact Steve Nesta x6386 with questions or concerns.

Af Little Elli Fon DWF

Dennis W. Ferrera

Vice President and Project Manager

Remediation, Industrial D&D, and Site Services

Attachment: As Stated

KLM:pvt

Orig. and 1 cc - Gary Morgan

DECEIVED NUN - 8 2005

ADMIN RECORD

ACTION ITEM STATUS:
PARTIAL/OPEN
CLOSED

ORIG. & TYPIST INITIALS: SMN:pvt

RF-46469(Rev.9/94)

Kaiser-Hill Company, L.L.C.

Rocky Flats Environmental Technology Site, 10808 Highway 93, Unit B, T707P, Golden, CO 80403-8200 ♦ (303) 966-7577

Steven H. Gunderson Colorado Department of Health and Environment 4300 Cherry Creek Drive South Denver, CO 80222-1530

RSOP FOR COMPONENT REMOVAL, SIZE REDUCTION, AND DECONTAMINATION ACTIVITIES NOTIFICATION LETTER FOR BUILDING 891 COMPONENT REMOVAL, SIZE REDUCTION, AND DECONTAMINATION

Mr. Gunderson:

In accordance with the Rocky Flats Cleanup Agreement Standard Operating Protocol (RSOP) for Component Removal, Size Reduction and Decontamination Activities, this letter and its attachments is notification for RSOP implementation. This notification is for all activities required to bring Building 891 to the unrestricted release criteria. This will involve component removal, size reduction and decontamination activities utilizing the methods specified in the RSOP. Additionally, a closure description like document is included to address the RSOP requirement for a Closure Plan that meets the substantive elements of a Closure Description Document.

Kaiser-Hill or a subcontractor will conduct this work. If Kaiser-Hill or the subcontractor would like to use a method or process not included in the RSOP then they are required to notify and, in consultation with DOE/LRA, the RFCA process for decision document modification will be used.

The appropriate checklists and information required by the RSOP are attached to this letter and should provide the necessary information. This work will be conducted in accordance with the work control documentation prepared by Kaiser-Hill or the subcontractor. The exact methods and process selected by Kaiser-Hill or the subcontractor and progress of the activities will be communicated to DOE/LRA through the consultative process, particularly the monthly RISS production meetings. The facility will not be breached during the activity.

As indicated in the RSOP, the LRA has 14 days to review the RSOP notification letter and provide feedback, including a definitive reason for not proceeding with the project. If no feedback is received within 14 days, the project will proceed as planned.

If you have any questions regarding this, please contact me at (303) 966-6246.

John Rampe U.S. Department of Energy

RSOP for Component Removal, Size Reduction, and Decontamination Activities Checklist

Project scope:											
Facility description: Consolidated Water Treatment Facility											
Description of pl	anned a		contamination, size r uilding 891 to the ur						al rec	quired to	
Facility/rooms/se	ts/area	s involved: All									
Is RCRA unit closure(s) part of the planned activity? If RCRA units are included, attach unit specific information sheets and drawings X No*											
		n Appendix A of the		X				mova	l/Siz	e	
Complete checklis	sts by ro	om/set/area/facility, as	appropriate		Redu						
	-		<u> </u>	X	Decon	tami	nation	<u> </u>			
RLCR Status	RLCR Status RLCR complete and concurrence received: N/A										
		RLCR initiated but i	ncomplete; concuri	ence	anticipa	ated:	N/A				
	X	RLC has not been ini	tiated¹ and is sched	luled	for initi	iation	on:	July 20	005		
		or initiated, what the work activities?	See attached Conta	ct Re	cord dat	ed 3/2	26/03				
Activity requires	modifi	cation to the ARARs li	sted in the RSOP.					Yes, letter		ch to	
	•						X	No			
Attach Admini	strative	Record file require	ements for the act	ivity	. Atta	chec	i				
Point of contact f	or each	facility/activity: Car	meron Freiboth								
Duration of work	activit	ies: 8 weeks	Anti	cipat	ed work	ς star	t: J	une 27,	2005		
Attach schedule	for each	facility or activity for	information purpo	ses.			_				
Attached											
•		removing contaminate on of the activity, conta	•		_			LRA co		tation required	
X No											

^{*} A closure plan is submitted pursuant to the RSOP for the Consolidated Water Treatment Facility, which meets the substantive elements of a closure description document.

¹ Evaluate using DPP, Sections 1.1.4 and 1.1.5 and the consultative process to implement activities

RSOP for Component Removal, Size Reduction, and Decontamination Activities Checklist

Arc	Are there deviations/exceptions to the RSOP for the proposed activity(ies)?										Yes						
	i										Х	No					
Provide an explanation of deviation/exception to the RSOP: Not applicable C. Check the appropriate resulting action box below									ole				-				
	Addi	tiona	IRFCA	decisio	n docum	ent re	quired (I	PAM –	IM/IRA)	•							
	Major modification to RSOP								Field cha	Field change to RSOP							
	Minor modification to RSOP L								LRA cons	LRA consultation							
Act	ivity(ies) will	result in	the fo	llowing w	aste t	ypes		•			Process	waste				
											X	Remedi	ation v	vaste			
	TRU	X	LLW	X	LLMW		Haz.	X	Sanitary	1	Othe	r: recyclab	le/re-u	se			
LR	A Notifi	catio	Review	Time		Х	14 days	, no R	CRA unit clo	sure	involve	d					
	•						30 days, RCRA unit closure involved										

FACILITY COMPONENT REMOVAL, SIZE REDUCTION, AND DECONTAMINATION ACTIVITY CHECKLIST

Building:

891

Closure Project Manager:

Cameron Freiboth

COMPONENT REMOVAL/SIZE REDUCTION

Component Type	,
Gloveboxes	
Tanks and ancillary equipment (located both inside and outside the facility)	Y
Fume hoods	
Ventilation/filtration systems (both inside and outside the facility)	>
Utilities and other equipment (both inside and outside the facility; including electrical, steam, and fire suppression systems)	>
Walls	
Floors	
Ceilings	
Roofs	
Other structural members	
Other*	

Removal/Size Reduction Technique	•
Small tools	~
Paving breaker, jackhammer and/or similar tools used to break up concrete	
Excavators, such as backhoes, to excavate underground components, such as tanks and ancillary equipment	
Hoists and cranes	
Plasma arc cutter	-
Diamond wire saw	
Wachs cutter	
Laser cutter	
Oxy-torch cutter	
Hydraulic shears	
Shear baler	
Water cutter using abrasives	
Arc air slice	
Arbor press	
Non-explosive cracking agent	
Other *	

* Describe "Other" Component Type(s) and/or Removal/Size Reduction Technique(s):

FACILITY COMPONENT REMOVAL, SIZE REDUCTION, AND DECONTAMINATION ACTIVITY CHECKLIST

DECONTAMINATION

Component Type:	V
Gloveboxes	
Tanks and ancillary equipment (located both inside and outside the facility)	~
Fume hoods	
Ventilation/filtration systems (both inside and outside the facility)	~
Utilities and other equipment (both inside and outside the facility; including electrical, steam, and fire suppression systems)	
Walls	~
Floors	~
Ceilings	
Roofs	
Other structural members	•
Other*	<u> </u>

Decontamination Technique	. Y
Wiping/scrubbing/washing with water or detergents	~
Vacuuming	•
Strippable Coating	
Grinding	
Scarifying	
Scabbling	
Paving breaker/chipping hammer	
Spalling	
Abrasive/grit blasting	
CO ₂ blasting	
Hydrolasing	
Strong mineral acids	
Organic or weak acids	
Additional oxidants, such as cerium and other similar metals	
Other *	

Prepared by:	Kimberly L. Myers	Date: June 1, 2005
	planned activity falls outside the scope of this RSO determine whether this RSOP should be modified to ould be written.	
	ter Component(s) and/or Decontainmatic	ni recimique(s).
* Describe !!Oth	ner" Component(s) and/or Decontaminatio	Other

Administrative Record Requirements for this Activity

- Final Rocky Flats Cleanup Agreement (RFCA)
- RFETS Decommissioning Program Plan (DPP)
- RFCA Standard Operating Protocol for Component Removal, Size Reduction, and Decontamination Activities
- Notification Letter and subsequent CDPHE correspondence, if appropriate
- Contact Records as applicable

Closure Plan
For the Consolidated Water Treatment Facility,
Building 891

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4.0	SPECIFIC CLOSURE ACTIVITIES	2
5.0	DISPOSTION OF CLOSURE WASTES	3
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7.0	AMENDMENT OF THE CLOSURE DESCRIPTION DOCUMENT	3

1.0 INTRODUCTION

1.1 Purpose and Scope

Part 3.4 of the Rocky Flats Closure Agreement (RFCA) Standard Operating Protocol for the Consolidated Water Treatment Facility (CWTF): Treatment and Disposal of Remediation Derived Wastewaters (November 2003), addresses closure of this unit. Closure of the unit is subject to the substantive requirements of a Closure Description Document, to address the plant's management of hazardous waste.

This Closure Plan applies to the treatment units in T900A, T900B, and in Building 891 proper.

2.0 METHOD OF CLOSURE AND PERFORMANCE STANDARD

The CWTF (at Building 891), described herein will be closed following guidance in Part X.D.2 for Unit Removal.

3.0 SYSTEM DESCRIPTION AND WASTE CHARACTERIZATION

The CWTF is located in the southeast area of the Industrial Area of the Site. Building 891 is a single story engineered metal structure. The building was built in 1991 and the early mission was treatment of groundwater from OU 1, located south and east of Building 881. In 1995, the treatment unit for OU 2 (T900A and T900B) was combined with the unit at Building 891. At this point in time the unit was renamed the CWTF, and through regulatory approval, began treating various waters generated at RFETS. At various times in the history of the CWTF, approval was given to treat water containing RCRA F-listed wastes.

At the inlet to the CWTF there is a Granulated Activated Carbon (GAC) unit to remove most organics that cannot be treated in the CWTF. T900A and T900B, the first portion of the treatment process are trailer-mounted chemical precipitation/microfiltration system designed primarily for the removal of metal contaminants. The solids were removed from the filter press, packed in drums, and disposed of as LLMW (EPA Codes F001, F002). However, after the unit treated groundwater from a remediation project conducted in 2004, contaminated with carbon tetrachloride, the solids also had the EPA code U211 applied. The liquid was pumped to a neutralization tank or recirculated through the precipitation/microfiltration system.

The next operation was the UV/peroxide oxidation unit where hydrogen peroxide was injected to oxidize the volatile organic compounds (VOCs). The liquid then passed through a GAC unit, which removed VOCs that were not adequately broken down by the previous treatment. After the GAC tank was a carbon dioxide injection system that converted metal sulfates to carbonates, to increase the efficiency of metals removal in the ion exchange treatment.

The ion exchange treatment system consisted of four ion exchange columns in series, with a degasification tower to remove carbon dioxide. The liquid first flowed into a strong base anion exchange column, which primarily removed uranium. The second step was a weak acid cation exchange column, which removed alkalinity associated with hardness. The degasification tower, next in line, removed carbonic acid produced as a byproduct of the weak acid column. The liquid then flowed into a strong acid

cation exchange column, which removed metals and excess hardness. The final step was a weak base anion exchange column for the removal of free mineral acidity.

The ion exchange unit was regenerated on a routine basis, with the cation units regenerated with hydrochloric acid and the anion units regenerated with sodium hydroxide. The waste stream, a combination of acid and base, was neutral for pH, but was normally characteristically hazardous for metals.

The treated effluent was then pumped into one of three effluent tanks, sampled and analyzed before release. If the effluent water met the standards, it was discharged directly to the South Interceptor Ditch. If the effluent water did not meet the effluent standards, it would be reintroduced to the treatment system.

4.0 SPECIFIC CLOSURE ACTIVITIES

Activities will be designed to achieve the closure performance standard, protect human health and the environment, and minimize waste. Specific work instructions will be developed prior to start of closure activities. These instructions will be developed in accordance with applicable RFETS policies and procedures.

5.0 DISPOSITION OF CLOSURE WASTES

The initial GAC unit, the chemical precipitation/microfiltration units in T900A and T900B, the UV/peroxide unit, and the next GAC unit will be disposed of as LLMW, with EPA codes F001, F002, and U211. The remainder of the treatment unit will be disposed of as LLW.

6.0 SOIL CONTAMINATION EVALUATION AND POST CLOSURE CARE

This unit has maintained an operating history (e.g., inspection logs, ECATS and occurrence reports) that indicates there have been no spills or releases to the environment as a result of waste managed activities in this unit. The closure activities for this unit will not impact the soils surrounding Building 891. Therefore, soil contamination will be evaluated as part of the building demolition and environmental restoration activities conducted under RFCA. Post-closure care activities are not necessary as part of the closure for this treatment unit.

7.0 AMENDMENT OF THE CLOSURE DESCRIPTION DOCUMENT

In conducting closure activities, unexpected events that are identified during implementation of closure activities may require an amendment to this RSOP Notification. Modifications to this RSOP Notification will be made in accordance with applicable regulations.

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D891104	B891 Alarm/Telecom Utility Isolations Wk Pckg	4	4	0	12JUL05	Ĭ	:			C					
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ROCKY FLATS ENVIRONMENTAL TECHNOLOGY SITE REGULATORY CONTACT RECORD

Date/Time:

3/26/03 - 1400 p.m.

Site Contact(s):

D. A. Parsons (D&D) – (DAP-014)

Phone:

(303) 966-6458⁸

Regulatory Contact: David Kruchek, CDPHE

Phone:

(303) 692-3328

Agency:

CDPHE

Purpose of Contact: Facility Anticipated Typing Reclassifications

Meeting Attendance

D. Parsons, RISS

D. Kruchek, CDPHE

J. Hindman, CDPHE

K. Wiemelt, K-H

E. Bryson, RFFO

S. Tower, RFFO

Discussion

During the weekly RISS Area Status meeting held on Wednesday afternoon, 3/26/03, Duane Parsons (RISS) discussed the attached proposed Facility Anticipated Typing Reclassification table. The attached table lists buildings that have not yet undergone a reconnaissance level characterization (RLC), and a justification for changing the anticipated facility Typing prior to the RLC. Based on a suggestion from Steve Tower (RFFO) several weeks ago, additional information was gathered on the buildings listed below, and the attached table was developed.

Based upon the additional information gathered, it was determined that some facilities should be changed from an anticipated Type 1 facility to an anticipated Type 2 facility, prior to the performance of the RLC. Likewise, based upon the additional information gathered, it was determined that some facilities should be changed from an anticipated Type 2 facility to an anticipated Type 1 facility, prior to the performance of the RLC.

Based on discussions of the attached table, it was determined that the following facilities should be changed from anticipated Type 1 facilities to anticipated Type 2 facilities prior to the performance of the RLC: Buildings 122, T122A, 891, T900A, T900B, and the 331 Garage. Refer to the attached table for the justifications for these re-typings. It was also discussed and agreed upon that any floor coverings and potentially contaminated equipment and/or systems that are not an integral part of these buildings (i.e., 122, T122A, 891, T900A, T900B, and the 331 Garage) will be removed from the buildings prior to the performance of the RLC. For example process waste drains embedded within the slab will remain; but carpet, floor tiles, loose equipment, and above-slab tanks and piping with potential low-levels of contamination will be removed prior to the RLC. Inprocess characterization will be performed prior to and during removal of the non-integral parts (e.g., floor coverings, equipment, systems, etc) of these buildings, as necessary to characterize this waste and to identify possible contamination in the buildings. Any elevated in-process characterization results will be provided to CDPHE and DOE. Once the floor coverings and potentially contaminated equipment and/or systems are removed. a combination RLC/PDS Type 2 characterization will be performed.

Based on discussions of the attached table, it was determined that the following facilities should be changed from anticipated Type 2 facilities to anticipated Type 1 facilities prior to the performance of the RLC: Buildings 664, 988A, 995-CCC-1, 995-CCC-2, 995-C-5, 995-EC1, 995-EC2, 995-EC3, 995-IC1, 995-IC2, 995-IC3, 790, 903A2, 906, 964, 569, and 570. Refer to the attached table for the justifications for these re-typings. It was also discussed and agreed upon that the Type 1 RLC of these buildings would be a more robust RLC than normal (i.e., more than the minimum amount of surveys and samples would be performed during the RLC of these buildings to ensure that adequate coverage is achieved in order to make appropriate final Typing and waste disposal decisions).

Based on discussions of the attached table, it was determined that Buildings 566 and 566A should be undergo additional in-process characterization surveys inside the ventilation ducting and remaining process waste piping. Then, based upon the in-process characterization surveys, evaluate if the 566 and 566A buildings should be reclassified to anticipated Type 1 facilities prior to the performance of the RLC. Once the Building 566 and 566A in-process characterization surveys are obtained, the results of the surveys will be presented to RFFO and CDPHE at a future date.

Based on discussions of the attached table, it was determined that the RLC of the 750 Pad Tents (Tents 2, 3, 4, 5, 6, 12 and 15) would be performed as a combination Type 2 RLC/PDS once all of the waste and equipment was removed from inside the tents (including the removal of the Tent 5 permacon). The 750 Pad Tents will remain as Type 2 facilities at least until the combination Type 2 RLC/PDS is completed.

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Based on discussions of the attached table, it was determined that since T664B and T664C buildings are reusable, portable, modified semi-trailers, that they could be unconditionally released utilizing the Property Release Evaluation (PRE) process. Additionally, since S750 building was a small, skid-mounted portable shed, it could also be unconditionally released utilizing the PRE process. Therefore, an RLC is not required for buildings T664B, T664C or S750.

Contact Record Prepared By: D. A. Parsons

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Additional Distribution:

C. J. Freiboth, K-H
F. Gibbs, K-H
D. Kruchek, CDPHE
S. Tower, RFFO
J. Hindman, CDPHE
M. Auble, K-H
D. Onyskiw, CDPHE
E. Bryson, RFFO
K. Wiemelt, K-H

The following two tables list buildings, and the justifications, for changing the "anticipated Typing" classification prior to the performance of the reconnaissance level characterization. Table 1 lists the facilities, and their justifications, for changing these buildings from "anticipated Type 1" to "anticipated Type 2" classifications. Table 2 lists the facilities, and their justifications, for changing these buildings from "anticipated Type 2" to "anticipated Type 1" classifications.

Table 1 - Change from Anticipated Type 1 to Type 2 classification

Facility	Justification
122	During the fires in the 1950's and 1960's, contaminated personnel spread contamination throughout the original portions of B122. There are three (3) process waste drains in B122.
	Although B122 should be Type 2, the characterization of B122 should be handled similar to how B441 was characterized. Since the high potential areas are on the floor and are currently covered by floor tile and/or carpet, these coverings should be removed prior to characterization. The characterization could then be done as a combination RLC/PDS. The later additions to B122 (i.e., the south and north additions) are not expected to be contaminated.
T122A	This portable decontamination trailer next to B122. The trailer was installed in 1997, and has been used to decontaminate wounded personnel. Although the decontamination sink or shower is not contaminated, the drain piping and under-trailer process waste tanks are potentially contaminated.
	Since the only likely potentially contaminated areas of T122A are the sink and shower drain piping and under-trailer tank, this equipment should be disconnected and removed prior to characterization. Once this equipment is removed, a combination RLC/PDS should be performed.
891	B891 has piping and tanks that are posted as internally rad contaminated due to treating wastewater with low levels of rad contamination. The 891 sump is posted as a contamination area; however, the posting applies to a removable fiberglass liner in the sump that can be easily removed. RCRA and/or CERCLA hazardous constituents may also be present in low levels inside the B891equipment.
	Since the levels of potential internal rad contamination in B891are very low (pCi/gram range), this equipment should be disconnected and removed prior to characterization. Once this equipment is removed, a combination RLC/PDS should be performed.
T900A	T900A has piping and tanks that are posted as internally rad contaminated due to treating wastewater with low levels of rad contamination. RCRA and/or CERCLA hazardous constituents may also be present in low levels inside the T900A equipment.
	Since the levels of potential internal rad contamination in T900A are very low (pCi/gram range), this equipment should be disconnected and removed prior to characterization. Once this equipment is removed, a combination RLC/PDS should be performed.
Т900В	T900B has piping and tanks that are posted as internally rad contaminated due to treating wastewater with low levels of rad contamination. RCRA and/or CERCLA hazardous constituents may also be present in low levels inside the T900B equipment.
	Since the levels of potential internal rad contamination in T900B are very low

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	(pCi/gram range), this equipment should be disconnected and removed prior to characterization. Once this equipment is removed, a combination RLC/PDS should be performed.
331 Garage	The garage portion of B331 used to be a metallurgical R&D laboratory during the 1950's and 1960's. Uranium and beryllium contamination were used and stored in B331 during this R&D period. There are three to four (3-4) process waste drains in B331.
	Although B331 should be Type 2, the characterization of B331 should be handled similar to how B441 was characterized. Since the high potential areas are on the floor and are currently covered by floor tile and/or carpet, these coverings should be removed prior to characterization. The characterization should then be done as a combination RLC/PDS. The later additions to B331 (i.e., Fire Department area) are not expected to be contaminated and will be treated as a separate facility.

Table 2 – Change from Anticipated Type 2 to Type 1 Classification

Facility	Justification
664	B664 was built in 1972 and has been used a waste storage, preparation, staging, and shipping facility. The facility was never utilized as a production facility, and never contained known un-encapsulated radioactive or hazardous materials. B664 is not listed as a "known beryllium area," nor is there any history of radioactive, RCRA/CERCLA, beryllium, or PCB spills. Routine rad surveys of the facility have shown no fixed or loose radioactive material. The only rad postings in the facility are radioactive material storage areas. There are no old or new process waste systems associated with B664.
	Once all of the radioactive waste containers are removed from the building, no residual radiological or non-radiological hazards should remain, except asbestos.
T664B and T664C	T664B and T664C are modified semi-trailers used to house real-time radiographic equipment for counting waste drums prior to shipment, and were brought onsite in 2001. The facilities were never utilized as production facilities, and never contained known un-encapsulated radioactive or hazardous materials. T664B and T664C are not listed as a "known beryllium area," nor is there any history of radioactive, RCRA/CERCLA, beryllium, or PCB spills. Routine rad surveys of the facilities have shown no fixed or loose radioactive material. The only rad postings in the facilities are radioactive material storage areas. There are no old or new process waste systems associated with T664B or T664C.
	Once all of the radioactive waste containers are removed from the trailers, no residual radiological or non-radiological hazards should remain.
S750	Building S750 is a 48 square-foot skid mounted portable shed acquired in the early 1990's. The shed has aluminum siding and an aluminum roof, the floor is wood. This shed has been used as a storage shed for non-hazardous and non-radiological operation such as the site housekeeping services, food service organization and site maintenance organization. There is no history of any radiological or hazardous operations in the facility. Routine rad surveys of the facility have shown no fixed or loose radioactive material. There are no old or new process waste systems associated with S750.

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Tents 2, 3, 4, 6, & 12

Tents 2, 3, 4, 6 and 12 were constructed in 1990 and have been used a waste storage, preparation, and staging facilities. The facilities were never utilized as a production facilities, and never contained known un-encapsulated radioactive or hazardous materials. Although the tents are on the "known beryllium area" list, routine surveys do not indicate the presence in the Tents. Minor spills have occurred on the Tent pad, but all spills were below reportable quantities and were cleaned up. Routine rad surveys of the facility have shown no fixed or loose radioactive material. The only rad postings in the facility are radioactive material storage areas. There are no old or new process waste systems associated with the Tents.

Once all of the radioactive waste containers are removed from the building, no residual radiological or non-radiological hazards should remain.

Note: Tent 5 contains a perma-con and will remain a Type 2.

988A, 995-CCC-1, 995-CCC-2, 995-C-5, 995-EC1, 995-EC2, 995-EC3, 995-IC1, 995-IC2, 995-IC3 Two waste streams are generated at the RFETS wastewater treatment plant, treated effluent and biosolids. For purposes of facility classification, those portions of the treatment process that have come into contact with the concentrated solids in the wastewater should be considered as anticipated Type 2 facilities (i.e., buildings 974 and 977, aeration basins 995-AB-1 and 995-AB-2; clarifier basins 955-C-1, 995-C-2, 995-C-3, 995-C-4, and digesters 995-D1 and 995-D2).

Units that come into contact with raw sewage and effluent only should be considered as anticipated Type 1 facilities (i.e., building B988A, chlorine contact basins 995-CCC-1 and 995-CCC-2, clarifier basin 995-C-5; effluent cells 995-EC-1, 995-EC-2, and 995-EC-3; and influent cells 995-IC-1, 995-IC-2, and 995-IC-3). Raw sewage may carry contaminants, but the concentration of solids is extremely low, generally less than 0.5%. As solids are concentrated in the treatment process through the clarifiers and digesters, there is the potential for contaminants to be concentrated.

All of these units should be reclassified as anticipated Type 1 facilities because they have only had contact with either raw sewage entering the treatment facility or treated wastewater just prior to release into the environment. Raw sewage is routinely analyzed for a number of operational parameters (pH, conductivity, suspended solids and others), and for a large suite of chemical parameters, including radionuclides, under various monitoring programs. There have been no recent incidents of contamination. The effluent is routinely monitored as well, and it routinely meets all requirements for release into the environment.

B988A is the final disinfection step and monitoring point on the discharged effluent. CCC1 and 2 are the chlorine contact chambers, which have been out of service for several years (chlorination disinfection was replaced with UV disinfection), and have never had contact with any portion of the solids waste stream. C-5 is the tertiary clarifier, which receives only effluent from the secondary clarifiers and no solids. EC-1, 2, and 3 are the effluent storage cells, which have only had contact with treated effluent from the facility. Finally, the IC-1, 2, and 3 units are the influent storage cells, which come into contact with raw sewage only.

The facilities were never utilized as production facilities. The facilities are not listed as a "known beryllium areas," nor is there any history of radioactive, RCRA/CERCLA, beryllium, or PCB spills. Routine rad surveys of the facilities have shown no fixed or loose radioactive material. There are no old or new process waste

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systems associated with these facilities. Once sewage treatment operations cease, there should no residual radiological or nonradiological hazards remaining. 790 Building 790 is a 6,768-sq. ft. single-story concrete building constructed in 1991. The building consists of three irradiation cells (A, B, and C) an instrument calibration support area, a control room, and an office area. Building 790 was designed and used as radiometric calibration facility. Specifically, it is used to expose thermoluminesent dosimeters (TLD) and calibrate site health physics instrumentation. This facility used and stored sealed sources and X-ray generating equipment. No hazardous chemicals are stored in Building 790, other than general cleaning supplies and small quantities (less than 1 pint) of alcohol and acetone to clean some instrument parts. The facility was never utilized as a production facility, and never contained known un-encapsulated radioactive or hazardous materials. B790 is not listed as a "known beryllium area," nor is there any history of radioactive, RCRA/CERCLA, beryllium, or PCB spills. Routine rad surveys of the facility have shown no fixed or loose radioactive material. The only rad postings in the facility are radioactive material storage areas. There are no old or new process waste systems associated with B790. Once all of the rad sources are removed from the building, no residual radiological or non-radiological hazards should remain. Sealed sources stored in Building 790 included, but are not limited to Pu, Am, Sr-90, Cf, Cs, Co-60, Ba, and Pm. 566 and 566A Building 566 is a single structure divided in to a 13,700 sq. ft. Site Alarm Maintenance and Respirator Repair Facility. Building 556 was originally constructed to be the site laundry facility (1991). The laundry was only operational for about 2 years, was never approved to handle the highly contaminated laundry, and only laundered two (2) loads of potentially contaminated low-level laundry and numerous loads of clean modesty clothing. Building 566 has always housed the Respirator Cleaning and Repair Group. In 1999, the Alarms Maintenance Servicing Center moved into the building. Alarm maintenance involves cleaning equipment, replacing faulty components, and testing and inspecting equipment. The Respirator Cleaning and Repair area contains a respirator washers, fume hoods, laundry carts, and radioactivity monitoring equipment. Detergent, bleach and water are used in the respirator washing process. Wastewater drains into two storage tanks located in the Building 566 pit and is then pumped to the sanitary drain system. Building 566 had above-slab process waste lines connected to the washing machines. These lines have since been removed along with the washing machines, and the only remaining line has been cut and capped near the NE outer wall of 566. Respirators and Alarm equipment are surveyed for radioactivity (and beryllium as necessary) prior to being transported to Building 566 to ensure no loose contamination exists. In the late 1990s, the B566 washers and dryers were removed and the waste trench under the washers was surveyed. Only very low levels of contamination were found in the trench and the areas were decontaminated (using power washer). Building 556A is the filter plenum for the laundry ventilation system in Building 556.

	It is 4,000 sq. ft. and was constructed in 1991. In the late 1990s, the air filter plenum stages was surveyed and no radiological contamination was found and thus the radiological postings were removed from the plenum. Several pieces of ventilation equipment and ducting leading to the 566A plenums still have internal rad contamination labels, however it is believed that this labels are no longer valid.
·	Based on the above information, and some additional in-process internal surveys of remaining ventilation equipment and process waste piping, it is very probable that 566 and 566A are not contaminated and could be reclassified to Type 1.
. 903A2	Building 903A2 is a 100 square-foot general storage shed acquired in 1993. This
	structure is a wood building with wood walls, wood floor and an asphalt shingle roof. This building sites on a concrete pad and is located west of the 903A Main
	Decontamination Facility (MDF). This building is used to store PPE and for general storage in support of the 903A MDF. There is no history of any radiological or
	hazardous operations in the facility. Routine rad surveys of the facility have shown no
	fixed or loose radioactive material. There are no old or new process waste systems associated with 903A2.
906	B906 was built in 1994 and has been used a TRU waste storage facility. The facility
900	was never utilized as a production facility, and never contained known un-
	encapsulated radioactive or hazardous materials. B906 is not listed as a "known
	beryllium area," nor is there any history of radioactive, RCRA/CERCLA, beryllium,
	or PCB spills. Routine rad surveys of the facility have shown no fixed or loose
	radioactive material. The only rad postings in the facility are radioactive material
	storage areas. There are no old or new process waste systems associated with B906.
	Once all of the radioactive waste containers are removed from the building, no residual radiological or non-radiological hazards should remain.
964	Building 964 is a 5,000 sq. ft. building and is currently identified as RCRA Unit 24. B964 was originally constructed in the mid-1960's and was used for general construction storage by a variety of site construction contractors. In 1986, the structure was modified for use as RCRA permitted Unit 24. These modifications include the installation of a spill containment system and the application of an epoxy concrete sealant. Ramps were installed to allow movement of containers in and out of the secondary containment system.
	The building currently stores solid wastes, but on occasions liquid waste has been stored in the building and was placed in metal secondary containment pans. Building 964 primarily stores solidified bypass sludge from Building 371. There have been no documented spills in B964.
	The facility was never utilized as a production facility, and never contained known unencapsulated radioactive or hazardous materials. B964 is not listed as a "known beryllium area," nor is there any history of radioactive, RCRA/CERCLA, beryllium, or PCB spills. Routine rad surveys of the facility have shown no fixed or loose radioactive material. The only rad postings in the facility are radioactive material storage areas. There are no old or new process waste systems associated with B964.
	Once all of the radioactive waste containers are removed from the building, no residual radiological or non-radiological hazards should remain, except asbestos.

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569	Building 569, also known as the Crate Counting Facility, is a 7,620 sq. ft. single-story building constructed in 1987. B569 contains radioactivity assay equipment and temporary waste storage operations. B569 is also RCRA Unit 59. Containers of low-level, low-level mixed, transuranic and transuranic mixed waste are received from throughout the plant site and assayed using a passive-active counter. Containers are surveyed prior being accepted into B569. Containers whose contents meet the disposal site waste acceptance criteria are transported to Buildings 664, 440, or 906 for storage pending off-site shipment. Those containers not meeting the disposal site waste acceptance criteria, or which exhibit physical damage or improper packing, are identified for repackaging and sent back to the originating building. No unpacking or repackaging is performed in B569.
:	The facility was never utilized as a production facility, and never contained known unencapsulated radioactive or hazardous materials. B569 is not listed as a "known beryllium area," nor is there any history of radioactive, RCRA/CERCLA, beryllium, or PCB spills. Routine rad surveys of the facility have shown no fixed or loose radioactive material. The only rad postings in the facility are radioactive material storage areas. There are no old or new process waste systems associated with B569.
	Once all of the radioactive waste containers are removed from the building, no residual radiological or non-radiological hazards should remain.
570	Building 570 is the filter plenum facility for the Crate Counting Facility (569) and is a 683 sq. ft. building constructed in 1987. B570 has never been activated and has never housed any radiological or hazardous operation. Ventilation ducting leading from B569 to B570 was never connected, and has always been blank-flanged off. Routine rad surveys of the facility have shown no fixed or loose radioactive material. There are no old or new process waste systems associated with B570.